

LA-UR-18-21915

Approved for public release; distribution is unlimited.

Title: Advancements in Fission Modeling for Nuclear Applications

Author(s): Jaffke, Patrick John

Intended for: Job Interview

Issued: 2018-03-08





Advancements in Fission Modeling for Nuclear Applications

Patrick Jaffke T-2 Postdoc, Los Alamos National Lab

UNCLASSIFIED



LA-UR-18-XXXXX

Collaborators

- P. Talou¹, I. Stetcu¹, T. Kawano¹, P. Möller¹, A. Sierk¹
- B. Byerly², L. Tandon², A. Hayes¹, G. Jungman¹
- S. Okumura³, S. Chiba³
- M. Devlin⁴, N. Fotiades⁴
- N. Schunck⁵, M. Mumpower¹

- ¹ Theoretical Division, Los Alamos National Lab
- ² Chemistry Division, Los Alamos National Lab
- ³ Tokyo Institute of Technology
- ⁴ Physics Division, Los Alamos National Lab
- ⁵ Nuclear Data and Theory Group, Lawrence Livermore National Lab





Slide 2

Outline

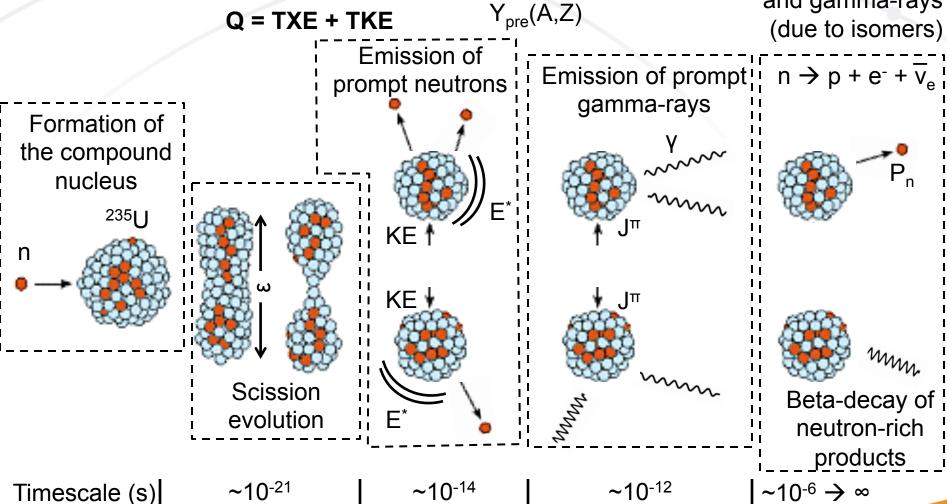
- Historical overview of nuclear fission
 - Current status of models and theory
- 2. Importance of fission modeling
 - Fundamental science, nonproliferation, criticality, heating, etc.
- 3. Applications:
 - Expanding Pu suite for multiplicity/criticality
 - Couple theory models to provide reasonable predictions
 - Creating new diagnostic tools with simplified depletion
 - New solutions in the very-low-burnup regime
- 4. Conclusions and outlook





What is the fission process?

Y_{post}(A,Z)
Delayed neutrons
and gamma-rays
(due to isomers)



UNCLASSIFIED



Outline

- 1. Historical overview of nuclear fission
 - Current status of models and theory
- Importance of fission modeling
 - Fundamental science, nonproliferation, criticality, heating, etc.
- 3. Applications:
 - Expanding Pu suite for multiplicity/criticality
 - Couple theory models to provide reasonable predictions
 - Creating new diagnostic tools with simplified depletion
 - New solutions in the very-low-burnup regime
- 4. Conclusions and outlook

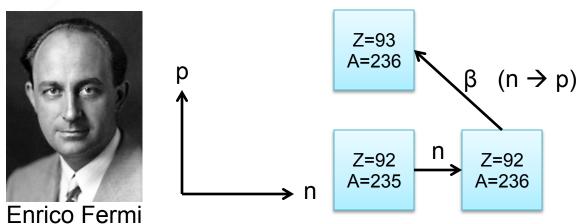




An accidental finding

1934: Fermi bombards Uranium with neutrons believing

he has produced heavier elements (Z=93)



Institute for Defense Analyses



Ida Noddack

Slide 6

amount to produce near neighboring elements. When heavy nuclei are bombarded by neutrons, it is conceivable that the nucleus breaks up into several large fragments, which would of course be isotopes of known elements but would not be neighbors of the irradiated element. Über das Element 93

> (1934)Dr.-Ing. Ida Noddack



2. Fission confirmed!







Fritz Strassmann



1938: Hahn and Strassmann <u>identify Barium</u> after n
 → Uranium

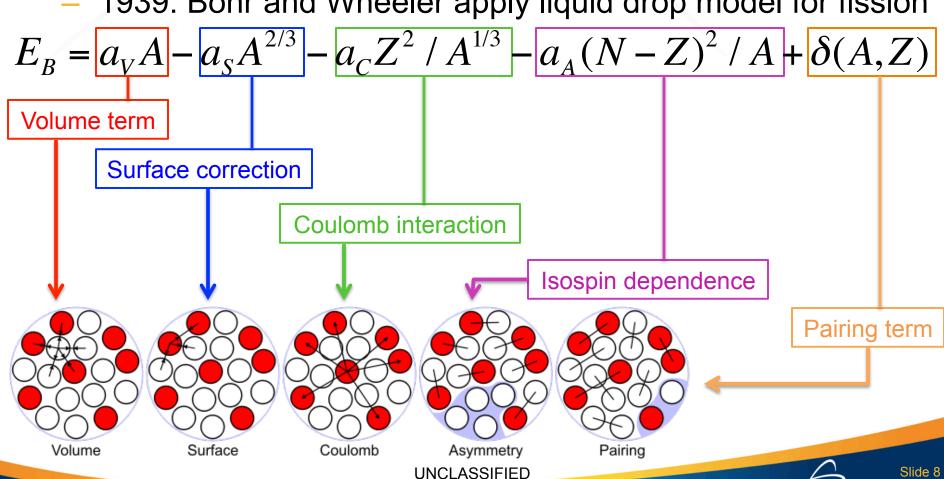
!! Roughly ½ the mass of Uranium !!

Disintegration of Uranium by Neutrons: a New Type of Nuclear Reaction (1939)





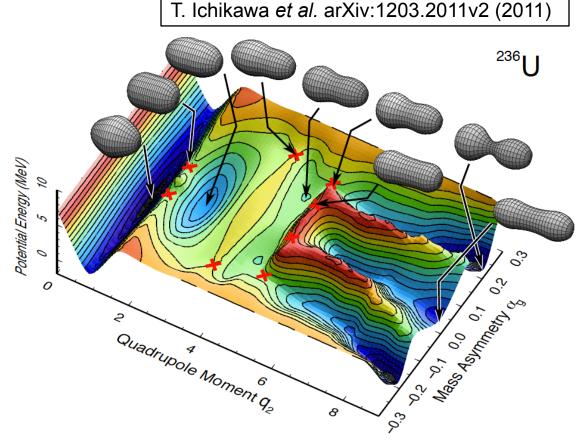






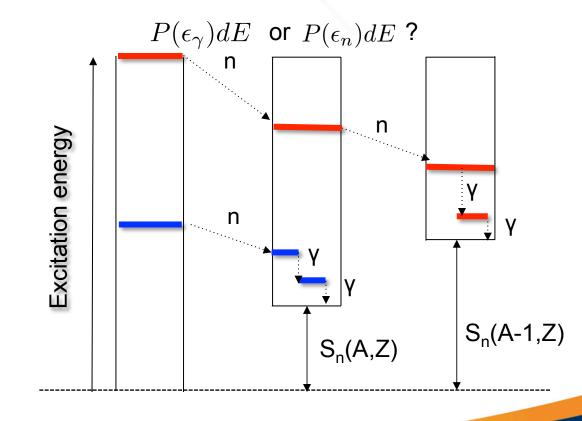


- 4. Scission Evolution (today)
 - Macro-micro: semi-classical (shape + nuclear corrections)
- Just a few model parameters fitted with nuclear masses
- Provides Y_{pre}(A,Z) for unmeasured reactions!!





- Fragment De-excitation (today)
 - <u>CGMF</u>: Monte Carlo implementation of Hauser-Feshbach
 - Requires starting distribution of fission fragments:
 - Y(A,Z,E*,Jπ)
 - A few model parameters
 - Provides prompt neutron and γ-ray emissions!!





UNCLASSIFIED



Outline

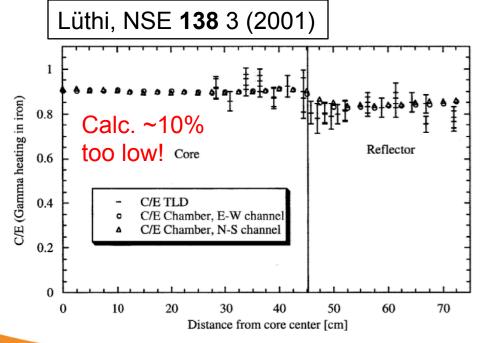
- Historical overview of nuclear fission
 - Current status of models and theory
- 2. Importance of fission modeling
 - Fundamental science, nonproliferation, criticality, heating, etc.
- 3. Applications:
 - Expanding Pu suite for multiplicity/criticality
 - Couple theory models to provide reasonable predictions
 - Creating new diagnostic tools with simplified depletion
 - New solutions in the very-low-burnup regime
- 4. Conclusions and outlook



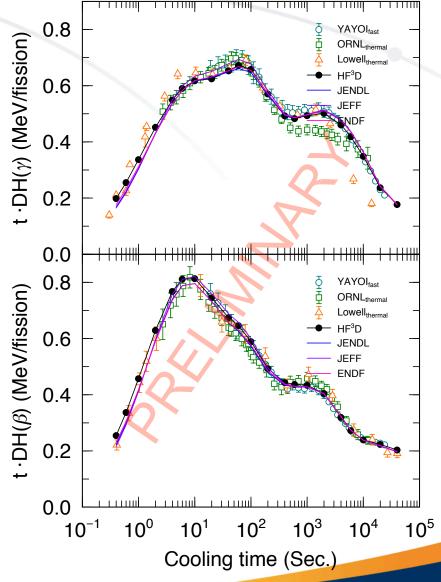


Some motivations

- Next-gen reactor design
 - Accurate energy release for new fuels, designs, etc.
 - Energy release from fragments, β's, γ-rays



Okumura, **PJ**, Kawano, Chiba, Talou



UNCLASSIFIED



Some motivations

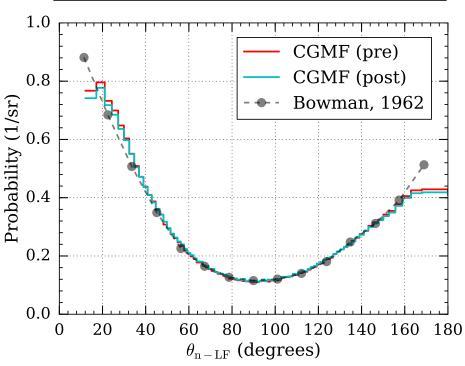
Verify trends in new correlated experimental data or guide new experimental designs

⁶Li glass to capture n Fission chamber

Chi Nu Array (LANL)

Neutrons predominantly emitted in direction of fragments (0° or 180°)

!! Direct impact on applications !!







Some motivations

- 3. Predictions for various applications
 - Multiplicity counting for source identification

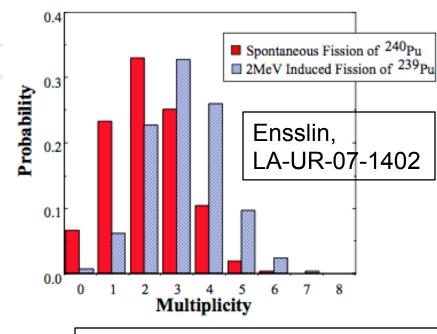
Multiplicity distributions different for Pu240sf and Pu239nf reactions!

Identification of isotopes from γ-ray spectroscopy

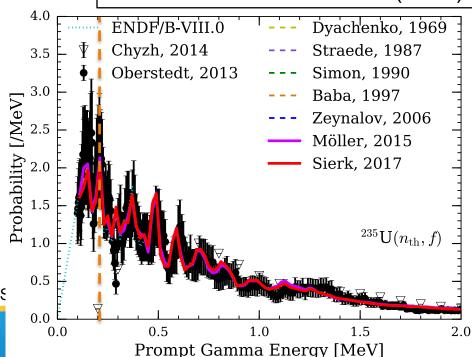
Fission product abundance related to intensity of specific γ-ray lines! 213 keV line corresponds to ¹⁰⁰Zr

UNCLASS









Outline

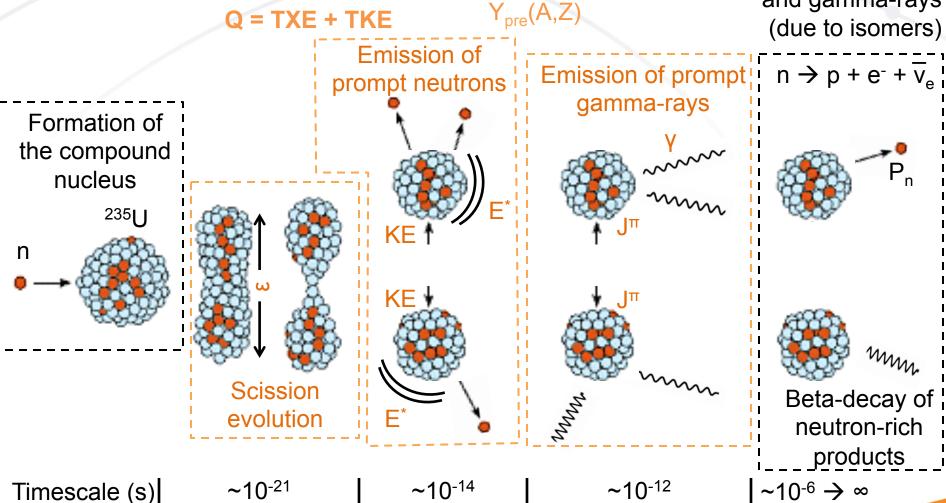
- Historical overview of nuclear fission
 - Current status of models and theory
- Importance of fission modeling
 - Fundamental science, nonproliferation, criticality, heating, etc.
- 3. Applications:
 - Expanding Pu suite for multiplicity/criticality
 - Couple theory models to provide reasonable predictions
 - Creating new diagnostic tools with simplified depletion
 - New solutions in the very-low-burnup regime
- 4. Conclusions and outlook





What is the fission process?

Y_{post}(A,Z)
Delayed neutrons
and gamma-rays
(due to isomers)



UNCLASSIFIED



Expanding the plutonium suite

Available data:

	_		
Isotope/Reaction	⊽ (n/fiss)	P _v	ε _n Spectrum
Pu236(sf)	Hicks (1965), Crane (1956)	Hicks (1956)	×
Pu238(sf)	Hicks (1965), Crane (1956)	Hicks (1956)	*
Pu238(n,f)	Jaffey (1970)	*	*
Pu239(n,f)	Frehaut (1980), Khokhlov (1976)	Holden (1988) Boldeman (1988)	Chatillon (2014), Nefedov (1983)
Pu240(sf)	Huanqiao (1984)	Boldeman (1985)	Gerasimenko (2002)
Pu240(n,f)	Khokhlov (1994), 73 Frehaut (1974)	*	Smith (1980)
Pu241(n,f)	Frehaut (1974), Vorobiyeva (1974)	Holden (1988), Boldeman (1985)	*
Pu242(sf)	Rodeman (1985)	Boldeman (1985)	Gerasimenko (2002)
Pu242(n,f)	Khokhlov (1994)	*	×

Expanding the plutonium suite

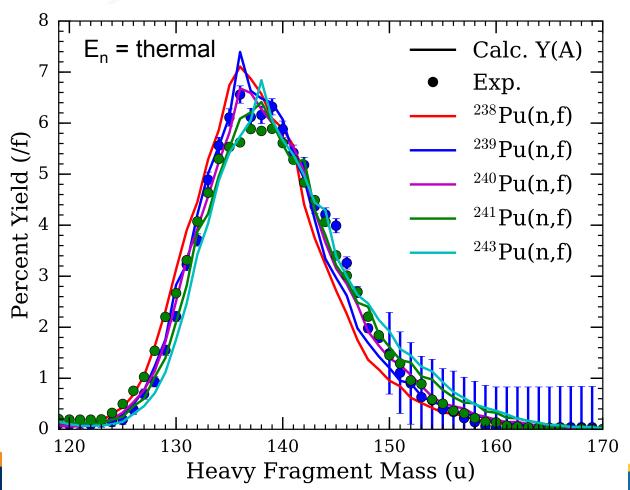
- Objective: provide reasonable estimates for prompt neutron multiplicity, distribution, and energies
 - And v, P_v , and ϵ_n depend on incident neutron energy E_n
- Issues:
 - CGMF requires Y_{pre}(A,Z,E*,J^π)
 - Need some mixture of theory + systematics guidance





Start with macro-micro Y_{pre}(A)

- Comparing theory with experiment
 - Reasonable agreement with data



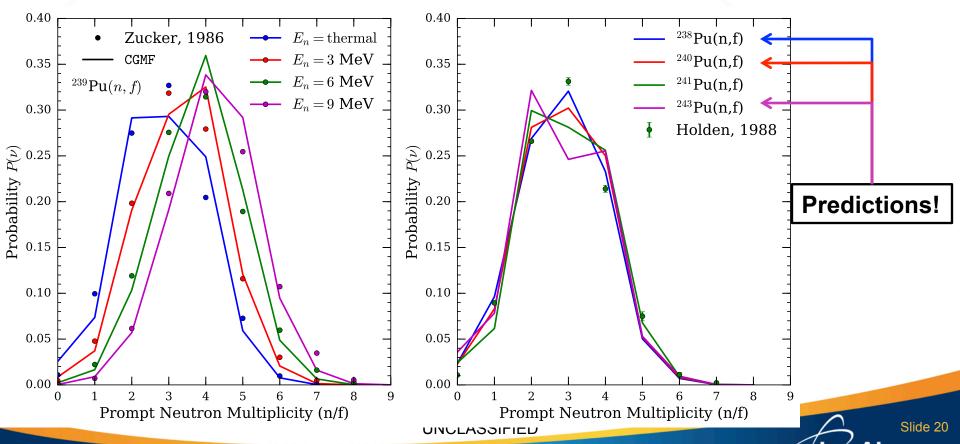
Use macro-micro Y_{pre}(A) as input for **CGMF** calculations Use Wahl's systematics (A. Wahl LA-13928) for the Y(Z|A) <TKE> systematics via Coulomb relation J[™] from Becker, PRC 87 014617 (2013)





Predictions for neutron characteristics

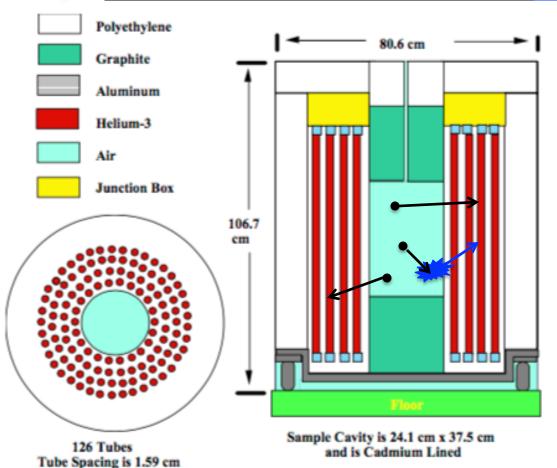
- P_v shows a similar energy-dependence
 - Shift towards higher neutron multiplicity with increasing E_n
 - Not a lot of data available though...





Neutron multiplicity counting problem

- Count multiplicity of neutrons in a time window
 - Relate singles/doubles/triples to effective ²⁴⁰Pu mass
 - Prompt neutrons from SF can induce fission in rest of Pu!



Can use calculated
 P_v for unmeasured
 fission reactions

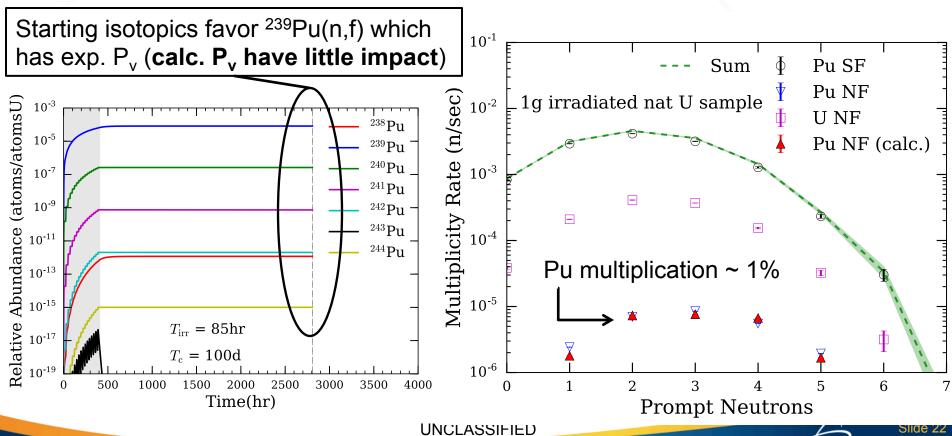
Estimate the multiplication factor

Ensslin, LA-UR-07-1402



Neutron multiplicity counting problem

- Get starting isotopics from depletion calculation
 - Determine spontaneous fission P_v rate
 - Calculate the additional P_v from neutron-induced reactions





Outline

- Historical overview of nuclear fission
 - Current status of models and theory
- Importance of fission modeling
 - Fundamental science, nonproliferation, criticality, heating, etc.

3. Applications:

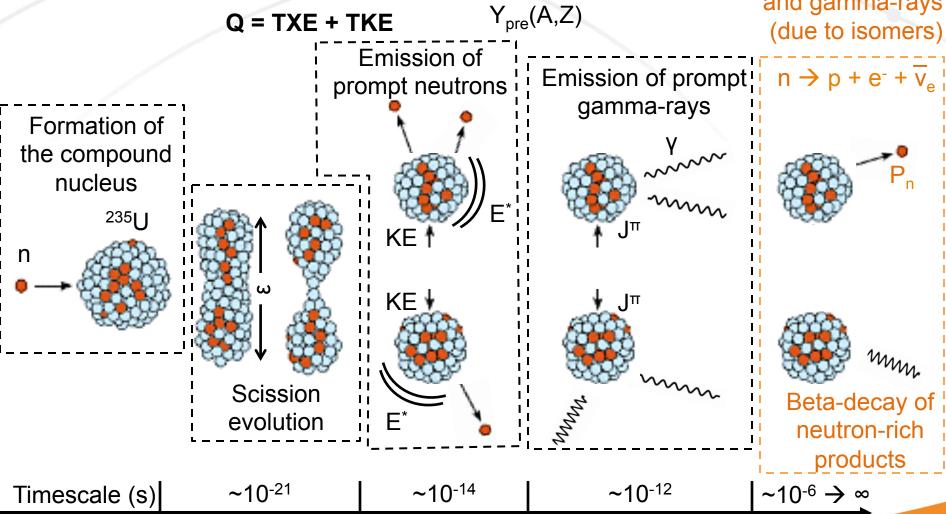
- Expanding Pu suite for multiplicity/criticality
 - Couple theory models to provide reasonable predictions
- Creating new diagnostic tools with simplified depletion
 - New solutions in the very-low-burnup regime
- 4. Conclusions and outlook





What is the fission process?

Y_{post}(A,Z)
Delayed neutrons
and gamma-rays
(due to isomers)







Some basic reactor physics:

238Am	239Am	240Am	241 А т	242Am
98 M	11.9 H	50.8 H	432.6 Ү	16.02 H
s: 100.00%	8: 99.99%	8: 100.00%	a: 100.00%	β-: 82,70%
0: 1.0E-4%	0: 0.01%	0: 1.9E-4%	SF: 4E-10%	8: 17,30%
237Pu	238Pu	23 9P บ	240Pu	241Pu
45.64 D	87.7 Y	24110 Y	6561 Y	14.329 Y
8: 100,00%	a: 100.00%	a: 100.00%	a: 100.00%	β-: 100.00%
0: 4.2E-3%	SF: 1.9E-7%	SF: 3.E-10%	SF: 5.7E-6%	α: 2.5E-3%
236Np	237 Np	238 Np	β 239 Np	240Np
153E+3 Y	2.144E+6 Y	2.117 D	2.356 D	61.9 M
_		_		
153E+3 Y 8: 86.30%	2.144E+6Y a: 100.00%	2.117 D	2.356 D	61.9 M

- Neutron exposure gives information about Pu production
- Cooling time dates the capabilities
- Depletion analysis uses reaction network to model the buildup of isotopes in a reactor environment

Primary fuel sources: LEU (~3%), nat U (0.71%)



UNCLASSIFIED



- Objective: determine the total neutron exposure and the age of reactor samples
 - Neutron exposure Φ_n relates to burnup
 - Cooling time of sample helps date the capabilities

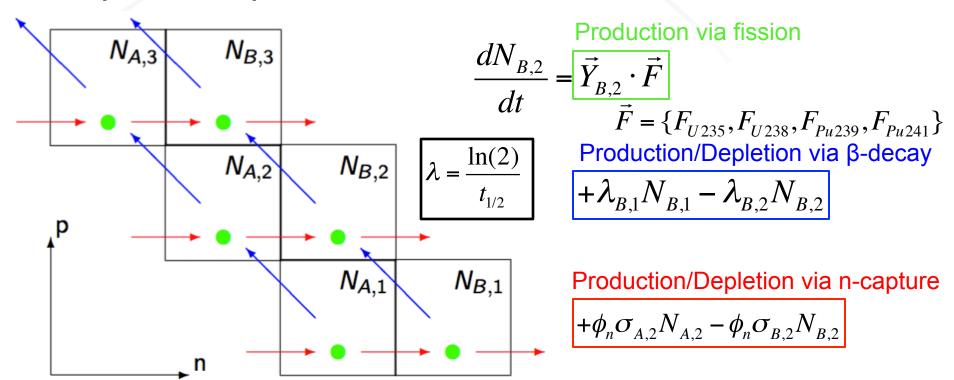
Issues:

- Typical methods are not ideal in very-low burnup
 - Not enough ²⁴¹Pu/²⁴¹Am produced for age estimate
 - Not enough ¹³⁴Cs or ¹⁵⁴Eu produced for measurement
 - Graphite Isotope Ratio Method (GIRM) too invasive
 - Additional problems if the samples are very old!





 Procedure: use <u>Bateman equations</u> to develop simplified depletion calculations



 Need decay constants (λ), cross-sections (σ), the fission rate (F), the fission yields (Y), and neutron flux (φ_n)





Example: Neutron exposure Φ_n from Uranium ratio

²³⁵U not produced in fission

$$\frac{dN_{U235}}{dt} = \vec{Y}_{U235} \cdot \vec{F}$$

No 235 Pa and $\lambda_{11235} \sim 0$

$$+\lambda_{Pa235}N_{Pa235}-\lambda_{U235}N_{U235}$$

$$+\phi_n\sigma_{U234}N_{U234}-\phi_n\sigma_{U235}N_{U235}$$

$$\begin{array}{c|c}
 & 235 \cup \\
 & -\sigma_{f} >> \sigma_{n} \\
\hline
 & N_{U235}(t) = N_{U235}^{0} e^{-\sigma_{U235} \phi_{n} t} \\
\hline
 & 238 \cup \\
 & -\sigma_{n} >> \sigma_{f} \\
\hline
 & N_{U238}(t) = N_{U238}^{0} e^{-\sigma_{U238} \phi_{n} t} \\
\hline
 & \mathcal{E} = \mathcal{E}_{0} e^{-\Phi_{n} (\sigma_{U235}^{T} - \sigma_{U238}^{T})}
\end{array}$$







- Cooling Time: Ratios of linear fission products
 - Linear in Φ_n
 - Long half-life w.r.t. T_{irr} and T_C
 - Short precursor halflives

$$Z_{Cs137} = Y_{Cs137} + Y_{Xe137} + Y_{I137}$$

$$T_{C} = \frac{1}{\lambda_{2} - \lambda_{1}} \ln \left(\frac{\alpha_{1/2} \lambda_{2} \vec{Z}_{2} \cdot \left\langle \vec{\Sigma}_{fiss} \right\rangle_{\Phi}}{\lambda_{1} \vec{Z}_{1} \cdot \left\langle \vec{\Sigma}_{fiss} \right\rangle_{\Phi}} \right)$$

$$\vec{Z} = \{Z_{U235}, Z_{U238}, Z_{Pu239}, Z_{Pu241}\} \qquad \langle \vec{\Sigma}_{fiss} \rangle_{\Phi}$$

Cumulative yields

UNCLASSIFIED

			•
	137Cs 30.08 Y	138Cs 33.41 M	139Cs 9.27 M
le l	β-: 100.00%	β-: 100.00%	β-: 100.00%
	K		
•	136 Xe >2.4E+21 Y 8.8573%	β 137 Xc 3.818 M	138 Xe 14.08 M
	2β-	β-: 100.00%	β-: 100.00%
	135I 6.58 H	136I 83.4 S	β 137I 24.5 S
	β-: 100.00%	β-: 100.00%	β-: 100.00% β-л: 7.14%

$$\alpha_{1/2} = \frac{\lambda_1 N_1}{\lambda_2 N_2}$$

Activity ratio of products 1 & 2

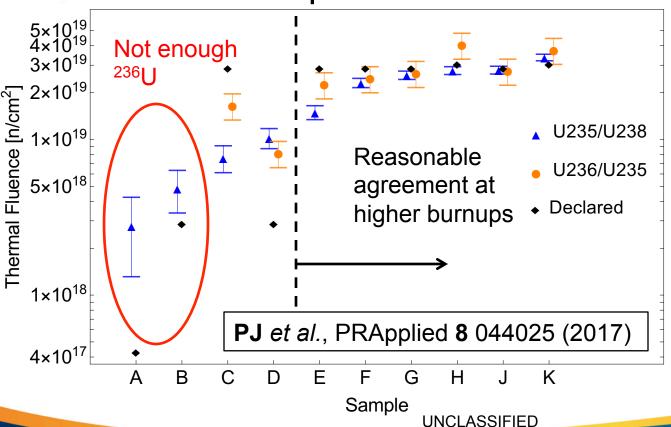
Flux-weighted macroscopic fission cross-section





Applying the diagnostics

- Start with U-metal and UO₃ archived samples
 - Separate Pu and U then TIMS for isotopics
- Uranium isotopics indicate nat U for ε₀

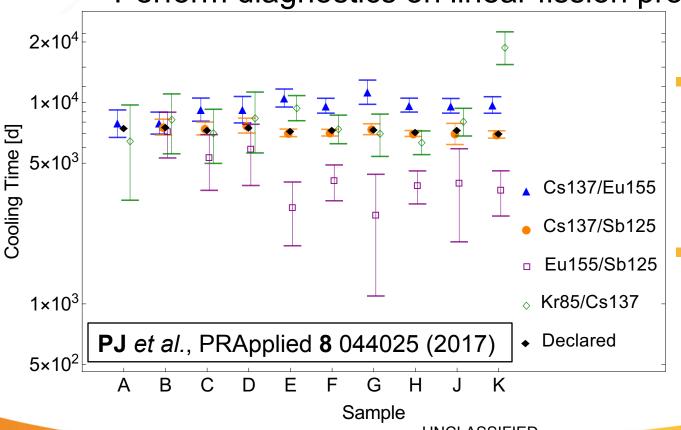






Applying the diagnostics

- Gamma spectrometry to identify fission products
 - Measured ⁸⁵Kr, ¹²⁵Sb, ¹³⁷Cs, ¹⁵²Eu, ¹⁵⁴Eu, ¹⁵⁵Eu
 - Perform diagnostics on linear fission products



Multiple T_C diagnostics required!

¹³⁷Cs/¹²⁵Sb matches declared T_C

Slide 31



Conclusion

- Fission is an extremely complex and intricate process
 - Tremendous modeling progress made so far
 - Stepping towards a <u>predictive fission model</u>: Y_{pre}(A) + CGMF
- Applications are far-reaching
 - Reactor heating, nonproliferation, forensics, fund. science
- Tools and models exist to fill in the gaps where experiments cannot or have not been yet
 - Ability to improve accuracy of applications
 - Find new applications and new designs





Next Steps

- 1. Fine-tune and perform optimization on Pu suite
 - Find <TKE> and d<TKE>/dE_n that produce reasonable neutron properties
 - Include calculations of spontaneous fission
- 2. Expand the U suite as well
 - Also useful for criticality, reactor heating, etc.
- Begin evaluation procedure to get consistent fission data for ²³⁵U, ²³⁸U, ²³⁹Pu
 - Identified as high-priority for nuclear data community



Thank you for your attention!

References & Funding

- PJ and P. Huber PRL 116 122503 (2016)
- A. Chyzh, **PJ**, et al. (submitted to PLB) (2017)
- PJ et al., PRApplied 8 044025 (2017)
- PJ, arXiv:1709.01183 (accepted in NSE 2018)
- PJ, P. Möller, A Sierk, P. Talou arXiv:1712.05511 (accepted in PRC 2018)
- S. Okumura, T. Kawano, P. Talou, **PJ**, S. Chiba arXiv:1802.01248 (2018)
- N. Fotiadis, PJ, et al. (in preparation) (2018)

U.S. Department of Energy (DOE) National Nuclear Security Administration (NA-22)





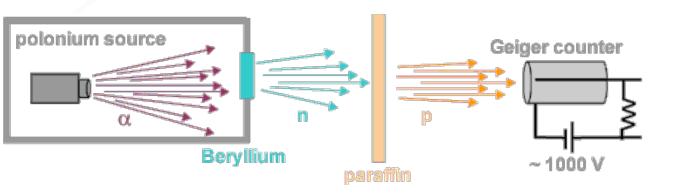
Slide 34

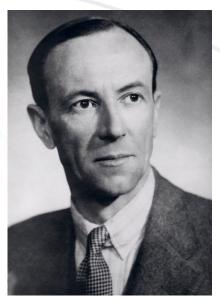
Extra



History of nuclear fission

- Gateway to Fission
 - 1932: Chadwick discovers the neutron





James Chadwick

Possible Existence of a Neutron

James Chadwick Nature, p. 312 (Feb. 27, 1932)

These results, and others I have obtained in the course of the work, are very difficult to explain on the assumption that the radiation from beryllium is a quantum radiation, if energy and momentum are to be conserved in the collisions. The difficulties disappear, however, if it be assumed that the radiation consists of particles of mass 1 and charge 0, or

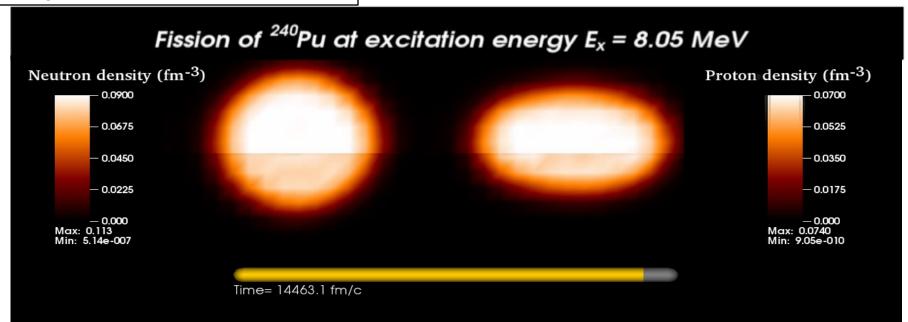
neutrons. The capture of the α -particle by the Be⁹ nucleus may be supposed to result in the formation of a C¹²



History of Nuclear Fission

- Nuclear Evolution (today)
 - Micro: based on nucleon-nucleon forces (calculate densities)
 - Computationally expensive (not ideal for yields)

Bulgac, PRL **116** 122504 (2016)

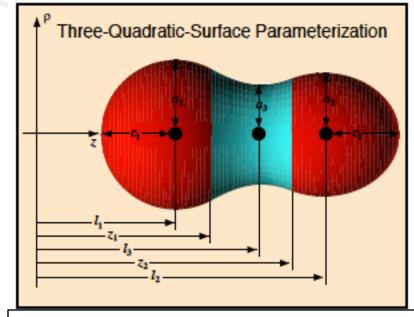






Macro-micro fission yields

- Compute the potential energy surface of a fissioning nucleus
 - Macroscopic shape + microscopic shell/pairing corrections
 - Macroscopic shape given by collection of shape variables q_i
 - 5D for $Y_{pre}(A)$ and 6D for $Y_{pre}(A,Z)$



P. Möller & T. Ichikawa EPJ A **51** 173 (2015)

Nuclear shape

$$E_T(Z,N,q_i) = \underbrace{E_M(Z,N,q_i)} + \underbrace{E_m^p(Z,N,q_i) + E_m^n(\mathbf{X},N,q_i)}_{\text{Loose dependence on N/Z}} + \underbrace{E_{odd}^p(Z,N,q_i) + E_{odd}^p(Z,N,q_i)}_{\text{Loose dependence on N/Z}}$$

Nucleon corrections





Hauser-Feshbach Model

$$P(\epsilon_{\gamma})dE \propto T_{\gamma}(\epsilon_{\gamma})\rho(Z, A, E - \epsilon_{\gamma})dE$$

 $P(\epsilon_{n})dE \propto T_{n}(\epsilon_{n})\rho(Z, A - 1, E - \epsilon_{n} - S_{n})dE$

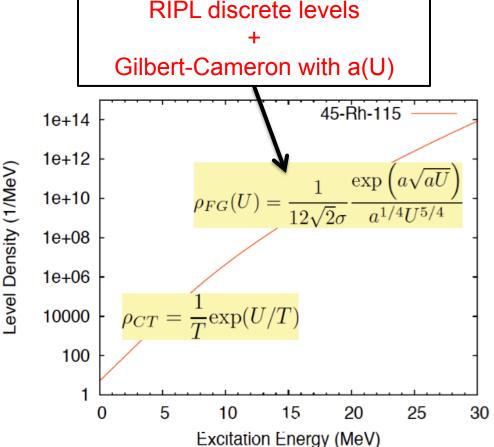
Need transmission coefficients and level densities

 $T_{\gamma}(\epsilon_{\gamma})$ from strength-function formalism

$$T^{XL}(\varepsilon_{\gamma}) = 2\pi f_{XL}(\varepsilon_{\gamma}) \varepsilon_{\gamma}^{2L+1}$$

 $\boldsymbol{T}_n(\boldsymbol{\epsilon}_n)$ from optical model calculations

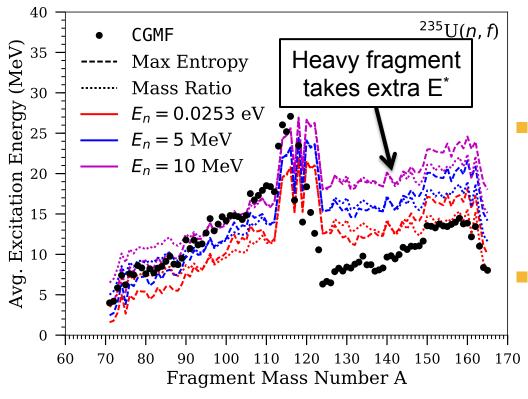
$$T_c = 1 - \left| \left\langle S_{cc} \right\rangle \right|^2$$



UNCL

Excitation Energy Sharing

Currently: vary R_T until v(A) matches (CGMF)



$$\frac{E_L^*}{E_H^*} = \frac{a_L}{a_H} R_T^2 \longrightarrow E_H^* = \frac{E_T^* a_H}{a_L R_T^2 + a_H}$$

- Next: share E_{int} via maximum entropy
 - Similar structure as fitted R_T(A)

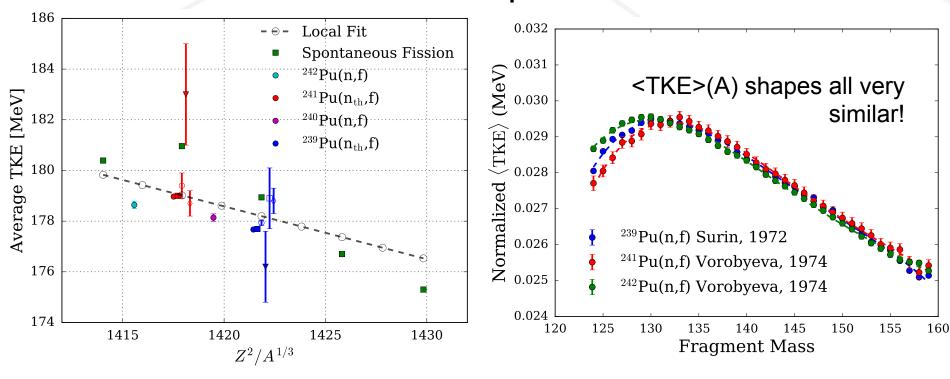
Max Entropy

$$\langle E_H \rangle = \int_0^{E_{\rm int}} \varepsilon \rho_H(\varepsilon) \rho_L(E_{\rm int} - \varepsilon) d\varepsilon / \int_0^{E_{\rm int}} \rho_H(\varepsilon) \rho_L(E_{\rm int} - \varepsilon) d\varepsilon$$





<TKE> systematics



- <TKE>(A) shapes from nearby Pu
- J^π distribution is a Gaussian

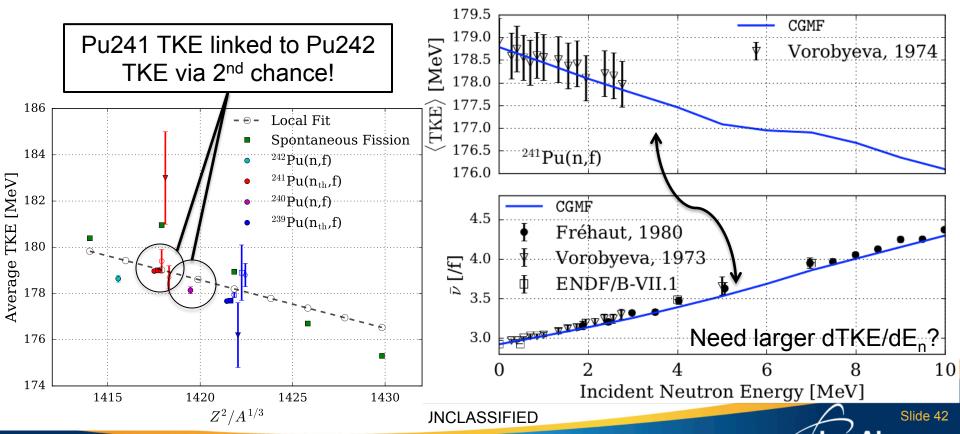
Becker, PRC 87 014617 (2013)





Neutron-<TKE> correlation results

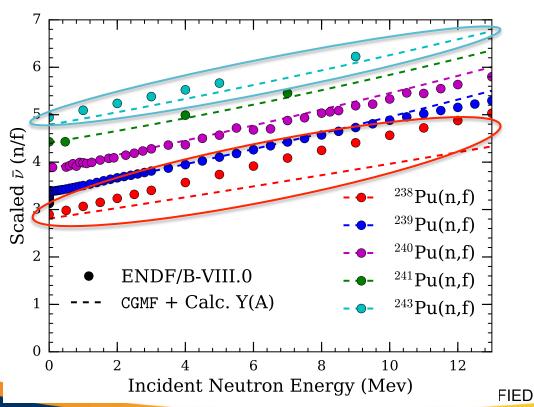
- More <TKE>, less <TXE> and fewer neutrons!
- Validate systematics with known Pu isotopes!
- Can use avg. n multiplicity to place constraints on <TKE>





Predictions for neutron characteristics

- Prompt neutron multiplicity agrees with ENDF/B-VIII
 - Shift differences could indicate <TKE> inaccuracy
 - Slope differences could be from bad d<TKE>/dE_n



We must be careful not to take ENDF as nature! Can often be evaluations/predictions another physicist performed!





Application – fission in r-process (FIRE)

0. 043 0. 041

0.039

0.037

0.035

0.033

0.030 0.028

0.026 0.024

0. 022 0. 020 0. 018

0.016

0.014

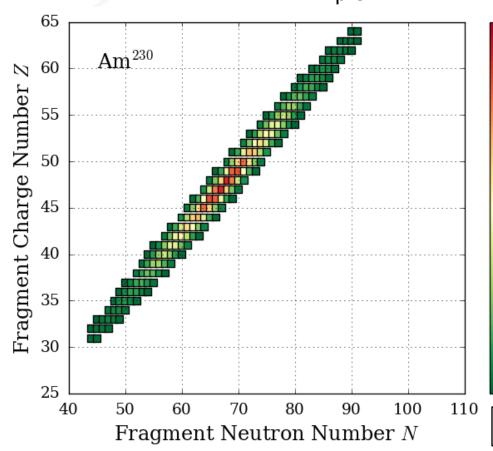
0.011

0.009 0.007

0.005 0.003

0.001

- Developing Y_{pre}(A,Z) systematics for r-process
 - Use macro-micro $Y_{pre}(A)$ and apply simple Y(Z|A)



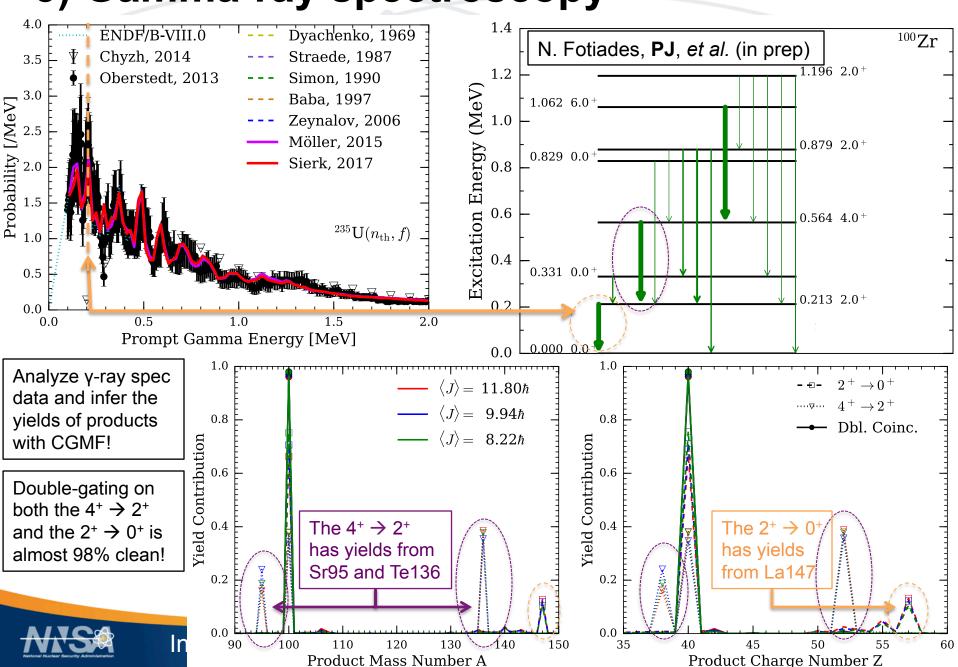
 Americium istopes show transition from symmetric to asymmetric

May need to include sf, βf, etc.

PJ, M. Mumpower, P. Möller (in prep)



3) Gamma-ray spectroscopy



Developing diagnostics for very-low burnup

Neutron Exposure: Uranium ratios

$$\Phi_n = \frac{\ln(\varepsilon_0 / \varepsilon)}{\sigma_{U235}^T - \sigma_{U238}^T}$$

$$\varepsilon = {}^{235}\text{U}/{}^{238}\text{U}$$

$$\Phi_n = \frac{1}{\sigma_{U235}^T - \sigma_{U236}^T} \ln \left(\frac{\sigma_{U235}^C - \rho(\sigma_{U236}^T - \sigma_{U235}^T)}{\sigma_{U235}^C} \right)$$

$$\rho = {}^{236}U/{}^{235}U$$

- 235[]/238[]-
 - Relies on knowing initial enrichment
- More accurate as concentration is higher
- 236**[**]/235**[**]-
 - Trouble when ²³⁶U is very low

Independent of initial enrichment





Deriving the cooling time diagnostic

Linear Systems: Simplest reaction networks

$$\frac{dN_L}{dt} = -(\lambda_L + \phi_n \sigma^T) N_L + \vec{Z}_L \cdot \vec{F}$$
 Depleted via β-decay and n-capture Produced via fission

For most linear nuclides, β-decay dominates...

$$\frac{dN_L}{dt} = -(\lambda_L + \phi_n \vec{o}^T) N_L + \vec{Z}_L \cdot \vec{F} \quad \xrightarrow{\text{Solution}} N_L(t) = \frac{\vec{Z}_L \cdot \vec{F}}{\tilde{\lambda}_L} (1 - e^{-\tilde{\lambda}_L t})$$

Taylor expand...

Assumes no initial abundance at start of irradiation → satisfied for linear systems

$$N_L(t) = \frac{\vec{Z}_L \cdot \vec{F}}{\tilde{\lambda}_L} (1 - [1 - \tilde{\lambda}_L t + \frac{1}{2} (\tilde{\lambda}_L t)^2 \dots]) \qquad \text{Add in } e^{-\lambda T} \text{ for decay time}$$

$$\downarrow \qquad \qquad N_L(t) = t(\vec{Z}_L \cdot \vec{F}) \longrightarrow \Phi_n \Big(\vec{Z}_L \cdot \Big\langle \vec{\Sigma}_{fiss} \Big\rangle \Big) e^{-\lambda_L T_C}$$

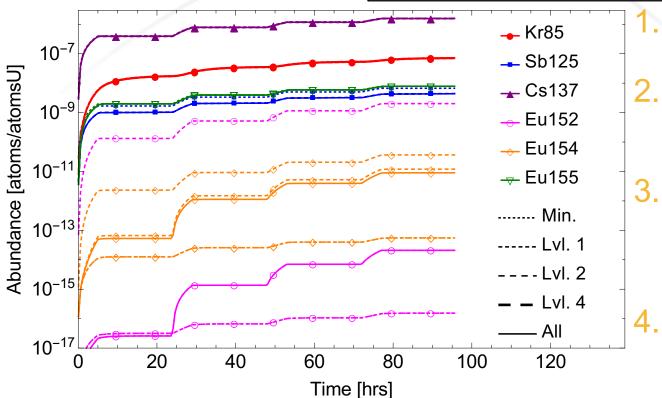




Defining linear systems

Linear Systems:

PJ et al., PRApplied 8 044025 (2017)



- β-parents are short-lived
- No significant neutron-capture channels
- Large cumulative yields (for measurement purposes)
 - Long-lived (for measurements purposes)

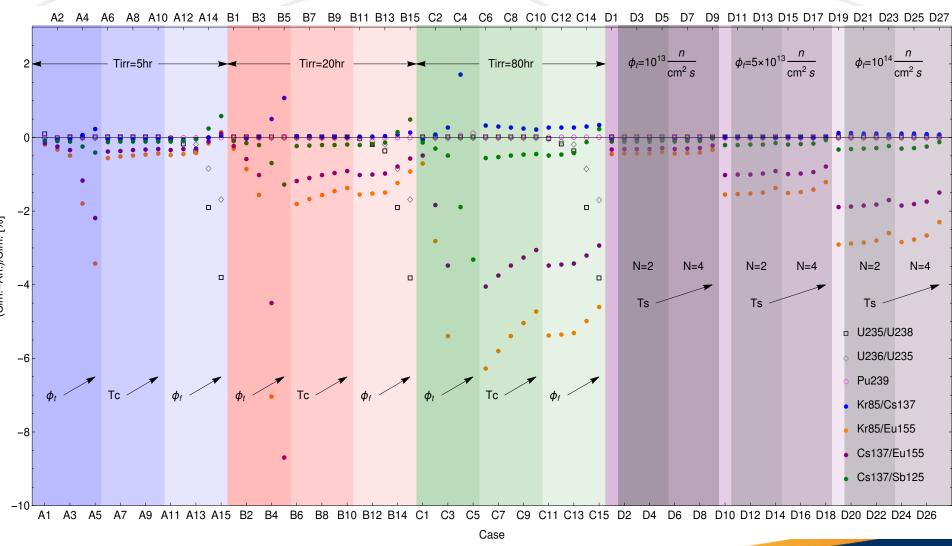
Nonlinear depend on size of reaction network! (152,154 Eu for example)

PJ and P. Huber PRL **116** 122503 (2016)





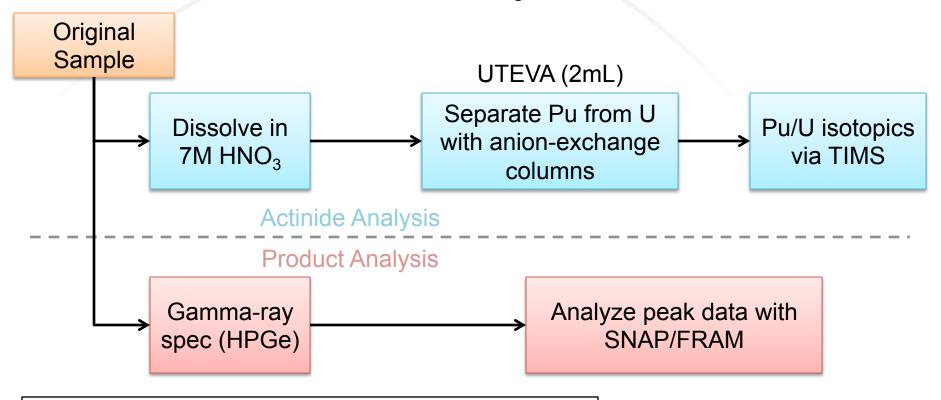
Verifying the diagnostics





Chemical separation

Start with U-metal and UO₃ archived samples



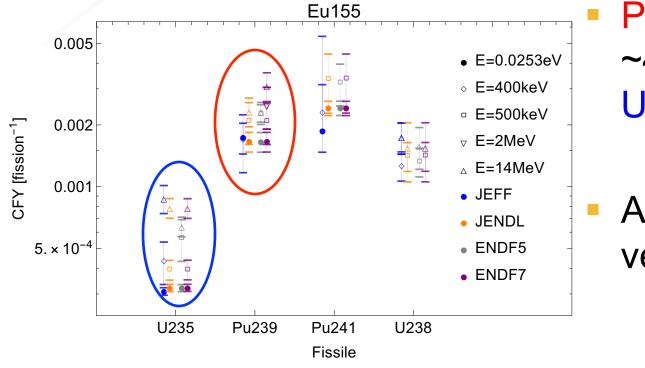
- B. Byerly et al., J. Radioanal Nucl Chem 307 (2016)
- L. Tandon et al., J. Radioanal Nucl Chem 282 (2009)





Why flux-averaging is needed

 Fission product yields change with fissioning isotopes and with energy (and data source!)



Pu239 yields are ~4x larger than U235 yields!

All databases are very similar

 Flux-average the fission rates so as not to bias towards U235 or Pu239 fissions



Eu155 issue

 T_c estimates using Eu155 show systematic disagreement with others

Abnormally large

Eu152, Eu154 abundances

nat Eu?

